



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

December 1, 2011

Mr. Mark A. Schimmel  
Site Vice President  
Prairie Island Nuclear Generating Plant  
Northern States Power Company - Minnesota  
1717 Wakonade Drive East  
Welch, MN 55089-9642

**SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 - ISSUANCE  
OF AMENDMENTS RE: REVISION OF TECHNICAL SPECIFICATION 5.2.1,  
"ONSITE AND OFFSITE ORGANIZATIONS" AND TS 5.3, "PLANT STAFF  
QUALIFICATIONS" (TAC NOS. ME5447 AND ME5448)**

Dear Mr. Schimmel:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 205 to Renewed Facility Operating License No. DPR-42 and Amendment No. 192 to Renewed Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 4, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML110390081).

The amendments make administrative changes to TSs 5.2.1 and 5.3 that: (1) allow certain requirements of onsite and offsite organizations to be documented in the Quality Assurance Topical Report (QATR); and (2) remove reference to specific education and experience requirements for operator license applicants.

M. A. Schimmel

- 2 -

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Thomas J. Wengert". The signature is fluid and cursive, with the first name "Thomas" and last name "Wengert" clearly distinguishable.

Thomas J. Wengert, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 205 to DPR-42
2. Amendment No. 192 to DPR-60
3. Safety Evaluation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY - MINNESOTA

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 205  
License No. DPR-42

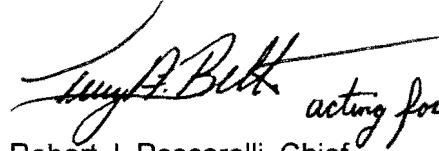
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), dated February 4, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 205 , are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Robert J. Pascarelli", followed by the words "acting for" in a cursive script.

Robert J. Pascarelli, Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed Facility  
Operating License and Technical  
Specifications

Date of Issuance: December 1, 2011



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY - MINNESOTA

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No.192  
License No. DPR-60

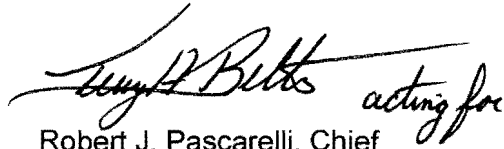
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), dated February 4, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192 , are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Robert J. Pascarelli", followed by the words "acting for" in a cursive script.

Robert J. Pascarelli, Chief  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed Facility  
Operating License and Technical  
Specifications

Date of Issuance: December 1, 2011

ATTACHMENT TO LICENSE AMENDMENT NOS. 205 AND 192

RENEWED FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Renewed Facility Operating License Nos. DPR-42 and DPR-60 with the attached revised pages. The changed areas are identified by a marginal line.

REMOVE

DPR-42, License Page 3  
DPR-60, License Page 3

INSERT

DPR-42, License Page 3  
DPR-60, License Page 3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

5.0-2  
5.0-5

INSERT

5.0-2  
5.0-5

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purpose of volume reduction and decontamination.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 205, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains



- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purposes of volume reduction and decontamination.

C. This renewed operating license shall be deemed to contain and is Subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains

## 5.0 ADMINISTRATIVE CONTROLS

### 5.2 Organization

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#### 5.2.1 Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for plant operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be defined and established throughout highest management levels, intermediate levels, and all operating organization positions. These relationships shall be documented and updated, as appropriate, in organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements, including the plant specific titles of those personnel fulfilling the responsibilities of the positions delineated in these Technical Specifications, shall be documented in the Updated Safety Analysis Report (USAR) or Quality Assurance Topical Report;
- b. The plant manager shall be responsible for overall safe operation of the plant and shall have control over those onsite activities necessary for safe operation and maintenance of the plant;
- c. A corporate officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety; and
- d. The individuals who train the operating staff, carry out health physics, or perform quality assurance functions may report to the appropriate onsite manager; however, these individuals shall have sufficient organizational freedom to ensure their independence from operating pressures.

5.0 ADMINISTRATIVE CONTROLS

5.3 Plant Staff Qualifications

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- 5.3.1 Each member of the plant staff shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, Revision 1, September 1975 except for the operations manager who shall meet the requirements of ANSI N18.1-1971, except that NRC license requirements are as specified in TS 5.2.2.e.
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed senior reactor operator (SRO) and a licensed reactor operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 205 TO RENEWED FACILITY

OPERATING LICENSE NO. DPR-42

AND AMENDMENT NO. 192 TO RENEWED FACILITY

OPERATING LICENSE NO. DPR-60

NORTHERN STATES POWER COMPANY - MINNESOTA

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

**1.0 INTRODUCTION**

By application dated February 4, 2011 (Agencywide Documents Access and Management System Accession No. ML110390081), Northern States Power Company, a Minnesota Corporation (NSPM, the licensee), requested changes to the Technical Specifications (TSs) for the Prairie Island Nuclear Generating Plant (PINGP), Units 1 and 2. The proposed changes would make administrative changes to TS 5.2.1 and 5.3 that: (1) allow certain requirements of onsite and offsite organizations to be documented in the Quality Assurance Topical Report (QATR); and (2) remove reference to specific education and experience requirements for operator license applicants.

Specifically, the proposed changes would revise PINGP, Units 1 and 2, TS Section 5.0, "Administrative Controls," by (1) revising TS 5.2.1(a) by adding the QATR as an allowable location for documentation of requirements for lines of authority, responsibility, and communications; and (2) revising TS 5.3.1 by the removal of the exception to operator license applicants' education and experience requirements, and the reference to a letter which references a specific industry guidance document. The proposed revision to TS 5.3.1 will update staff qualifications and make them consistent with the licensee's other operating nuclear power plant (Monticello Nuclear Generating Plant (MNGP)).

**2.0 REGULATORY EVALUATION**

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. In Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, the Commission established regulatory requirements related to the content of TSs. This regulation requires that the TSs include items in eight specific categories. These categories include 1) safety limits, limiting safety systems settings, and limiting control

Enclosure

settings, 2) limiting conditions for operation (LCO), 3) surveillance requirements (SRs), 4) design features, 5) administrative controls, 6) decommissioning, 7) initial notification, and 8) written reports. The regulation does not specify the particular requirements to be included in a plant's TSs. However, the Nuclear Regulatory Commission (NRC) has approved Standard Technical Specifications (STS) for each reactor type and encourages licensees to upgrade their TS consistent with the regulations and conforming, to the extent practicable, to Revision 3 with incorporated Travelers to the improved STS.

The NRC developed criteria, as described in the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (58 FR 39132), to determine which design conditions and associated surveillances should be located in the TSs as LCOs. Four criteria were subsequently incorporated into the regulations by an amendment to 10 CFR 50.36 (60 FR 36953).

The Commission's Final Policy Statement and documentation related to the revision of 10 CFR 50.36 acknowledged that implementation of the four criteria may permit some requirements presently in the TSs to be relocated to other licensee-controlled documents and programs.

Licensees may propose revisions to the TS to adopt STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The NRC staff reviews such proposals and decides whether or not to change the licensee's TS.

On December 12, 1995, the NRC issued Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance," to inform licensees of experiences involving the relocation of TS administrative controls related to quality assurance. In AL 95-06, the NRC staff identified typical TS administrative controls that may be relocated to licensee quality assurance control documents subject to the controls of 10 CFR 50.54(a). TS administrative control requirements identified in AL 95-06 include the independent safety engineering group, reviews and audits, the procedure review process, and records and record retention.

The NRC staff reviewed the TSs proposed to be relocated for compliance with 10 CFR 50.36 and agreement with staff guidance as established in NUREG-1431, Rev. 3, "Standard Technical Specifications, Westinghouse Plants," for Pressurized Water Reactors (PWRs).

On March 20, 1985, the NRC issued the Commission Policy Statement on Training and Qualification of Nuclear Power Plant Personnel, which endorsed the National Academy for Nuclear Training (NANT) and the training accreditation process. In Generic Letter 87-07, "Information Transmittal of Final Rulemaking for Revisions to Operator Licensing - 10 CFR 55 and Conforming Amendments," dated March 19, 1987, and in NUREG-1262, "Answers to Questions at Public Meetings Regarding Implementation of Title 10, Code of Federal Regulations, Part 55 on Operators' Licenses," published November 1987, the NRC indicated it would accept a facility's licensed operator training program if the facility certified in writing that

the program was accredited and based on a systems approach to training (SAT). This certification would supersede the requirements of ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel," and ANSI/ANS-3.1-1978, "Selection, Qualification and Training of Personnel for Nuclear Power Plants." Facility licensees were advised to submit a request to the NRC for an administrative change to their licensing documents to revise or delete, as appropriate, the requirements that had been superseded.

In 1992, the NRC published its proposed rule 10 CFR 50.120, "Training and Qualification of Nuclear Power Plant Personnel." The proposed rule stated that, if adopted, the rule would supersede the Policy Statement on Training and Qualification of Nuclear Power Plant Personnel and would not result in any change to accredited programs. The Commission concluded that accredited programs, implemented consistent with industry objectives and criteria, would be in compliance with this regulation. In April 1993, the NRC published its final rule on training and qualification of nuclear power plant personnel. The rule requires nuclear power plant licensees to establish, implement, and maintain SAT-based training programs for nine categories of nuclear power plant personnel. Accreditation of these nine training programs is an acceptable means of meeting the requirements of 10 CFR 50.120.

On January 18, 2001, the NRC published NRC Regulatory Issue Summary (RIS) 2001-01, "Eligibility of Operator License Applicants," to familiarize licensees with the NRC's current guidelines for the qualification and training of operator and senior operator license applicants. RIS 2001-01 acknowledged that 10 CFR 55.31(a)(4) allows the NRC to accept an application for an operator's license if the facility licensee certifies that the applicant has successfully completed a Commission approved training program that is based on SAT. In addition, RIS 2001-01 stated that: (1) a training program would be considered approved by the NRC when it receives or renews accreditation from the National Nuclear Accrediting Board (NNAB); (2) accreditation of operator training programs suggests that facilities are implementing the education and experience guidelines endorsed by the NNAB; (3) NANT guidelines for education and experience (those in effect in 1987 or those issued in January 2000) outline acceptable methods for implementing the Commission's regulations; and (4) the staff encourages all facility licensees to review their requirements and commitments related to operator and senior operator education and experience and to update their documentation (e.g., Final Safety Analysis Report, Technical Specifications, and training program descriptions) to "enhance consistency and minimize confusion."

### 3.0 TECHNICAL EVALUATION

#### 3.1 Proposed Changes to TS 5.2.1, Onsite and Offsite Organizations

Section 5.0 of the PINGP TSs contain administrative controls requirements in accordance with 10 CFR 50.36(c)(5). In its application, the licensee proposed to revise TS 5.2.1(a) by including the QATR as an allowable location for documenting the requirements for lines of authority, responsibility, and communications. The licensee stated that the addition of the QATR would allow a document common to PINGP and MNGP to contain the organizational relationships per TS 5.2.1(a).

Section 50.36(c)(5) of 10 CFR states, "administrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting

necessary to assure operation of the facility in a safe manner." TS 5.2.1 contains the requirement for establishing onsite and offsite organizations and positions for activities affecting safety of the nuclear plant. Specifically, 5.2.1(a) establishes the documented location of lines of authority, responsibility, communications, and specific titles of the personnel.

The NRC staff has reviewed the licensee's proposed changes to TS 5.2.1(a) against the criteria for administrative controls set forth in 10 CFR 50.36(c)(5). The NRC staff has determined that the content of the proposed changes involve administrative controls related to quality assurance requirements. In addition, the NRC staff determined that the addition of the wording "or Quality Assurance Topical Report" is editorial in nature and does not change the meaning of the intended requirements. The relocation requirements will be directly controlled by regulations in accordance with 10 CFR 50.54(a) or 10 CFR 50.59. Further, the NRC staff determined that the proposed changes are consistent with the requirements identified for relocation of the quality assurance plan by AL 95-06 and are also consistent with the guidance in NUREG-1431. Therefore, the NRC staff concludes that the proposed change to TS 5.2.1(a) is acceptable.

### 3.2 Proposed Changes to TS 5.3, Plant Staff Qualifications

In its application, the licensee proposed to revise PINGP Units 1 and 2 TS 5.3.1, by removing the exception to operator license applicants' education and experience requirements, and the reference to a letter which references a specific industry guidance document. The proposed revision to TS 5.3.1 will update staff qualifications and make them consistent with the licensee's other operating nuclear power plant (MNGP).

The revision proposed by NSPM will revise PINGP Units 1 and 2 TS Section 5.3, "Plant Staff Qualifications," to remove outdated licensed operator educational and experience requirements, based on NNAB's accreditation of PINGP's licensed operator training programs. This proposed change is consistent with 10 CFR 55(a)(4). In addition, this change will not affect the 10 CFR 50.36(c)(5) requirement to ensure the licensee maintains administrative controls that assure the operation of the facility in a safe manner. Therefore, the NRC staff concludes that the proposed change to TS 5.3.1 is acceptable.

## 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: S. Anderson, NRR  
S. Currie, NRR

Date: December 1, 2011



M. A. Schimmel

- 2 -

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Thomas J. Wengert, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 205 to DPR-42
2. Amendment No. 192 to DPR-60
3. Safety Evaluation

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